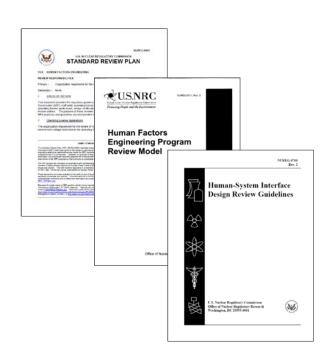
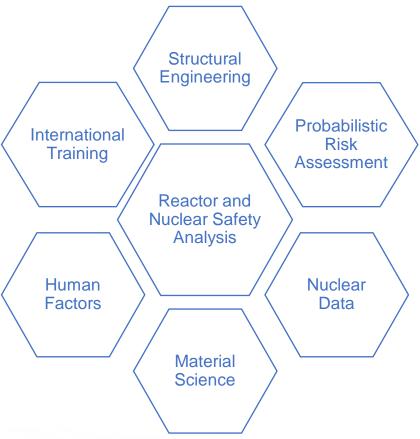
Nuclear Science and Technology Department Capabilities





Nuclear Science and Technology Department Capabilities







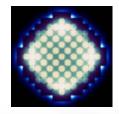


BNL Nuclear Energy Programs

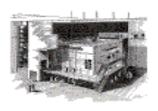
- NNDC 1952*
- Technical Support for NRC 1974*
- International Nuclear Safety Program 1986*
- Accelerator Transmutation of Waste 1991
- Proliferation Resistant Reactors 1996*
- Advanced Nuclear Fuels*
- Materials in Radiation Environments*
- * Continuing program









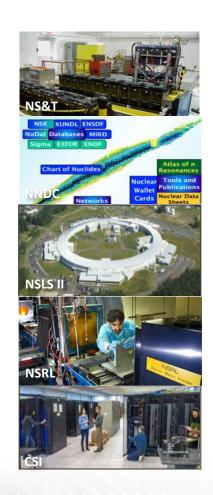








Advanced Reactor Development Support



Relevant Capabilities

- Radioactive materials characterization
- Reactor simulations
- Control room design/human factors
- Reactor licensing
- International commercialization

Partners/Joint Appointments

University

National Lab

Industry



BNL/DOEFacilities























National Nuclear Data Center

Nuclear Science References (NSR)

All nuclear physics articles indexed according to content

EXFOR

Compiled nuclear reaction data

XUNDL

Compiled nuclear structure and decay data

ENSDF

Recommended nuclear structure and decay data

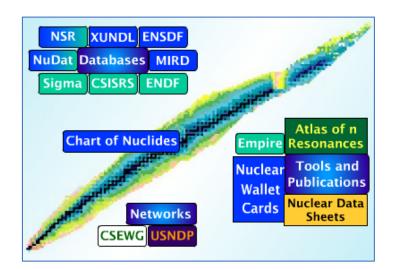
ENDF – Data needed in many applications: design, operation, and decommission nuclear reactors. Recommended particle transport and decay data with a strong emphasis on neutron-induced reaction data

Nuclear Data Sheets

Journal devoted to the publication of nuclear data articles

Web dissemination

www.nndc.bnl.gov



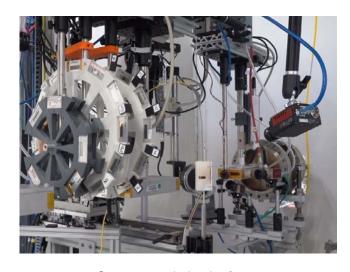


Molten Salts in Extreme Environments

Mission: To provide fundamental and predictive understanding/models of molten salt chemistry, including effects of solutes, impurities, and radiation

- Physics and chemistry of molten salts
- Interactions with solutes (actinides, fission products, corrosion products) and radiation
- Interactions with and degradation of reactor materials

The experimental techniques developed can be applied to real reactor systems – engineering alloys and salts



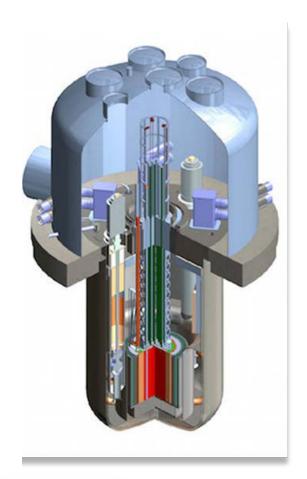
Custom made in situ furnace for molten salt studies (25°C to 700°C)





Supporting NRC Reactor Licensing

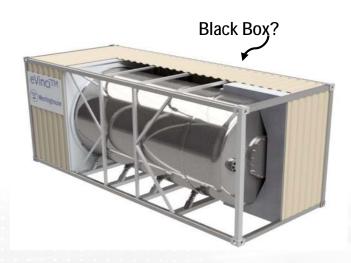
- ➤ Technical support on safety and licensing issues for power, research, test, and special purpose reactors.
- Reviews of licensee submittals (e.g., design certification, topical reports, life extension).
- ➤ Lead on many Phenomena Identification and Ranking Tables (PIRTs).
- ➤ Support for PRA Level 2 & 3 studies, nuclear power plant accident consequence analysis and emergency preparedness.
- Review of industry standard for safety-related composite modular and steel structures.
- Human factors issues.
- International regulatory support.

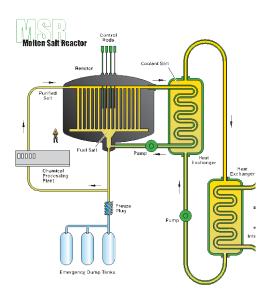




Advanced Reactor Analysis Activities

- Research phenomena important in Modeling and Simulation of advanced reactor designs
- Technical and regulatory issues of microreactors
- Regulatory impact of advanced reactor designs



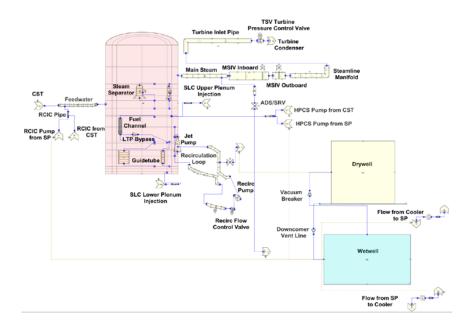






Reactor Safety Support

- Support NRC in revising the Standard Review Plan (NUREG-0800)
- Design reviews of advanced designs
- LOCA methodology for PWR and BWR
- Accident tolerant fuel fuel and cladding
- Extensive model development to perform coupled transient neutronics and thermalhydraulic analysis
- Research reactor core conversion (LEU to HEU)







PRA Methodology, Applications, and Guidance

PRA Methodology Development

- Level 1, 2, and 3; full power and shutdown PRAs
- Methodology of PRA applications

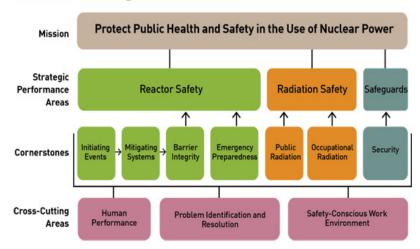
Risk-informed Technical Specifications

- Handbook of Methods for Risk-based Analyses of Technical Specifications – 1994
- Development of AP1000 Standard Technical Specifications

Risk-informed Inspections and Reactor Oversight Process

- Plant-specific models for Significance Determination Process
- Technical basis for At-Power Significance Determination Process

Reactor Oversight Framework

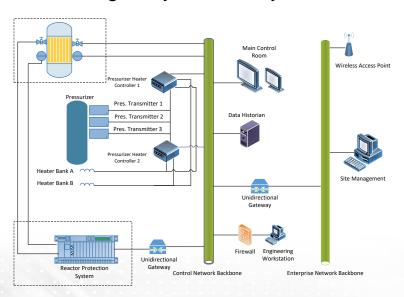


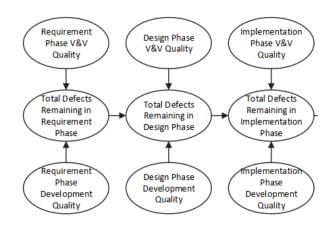




Digital I&C System Reliability

- Quantify the probability of digital system failure caused by software bugs through statistical testing
- Estimate the probability of software failure on demand-based quality of the development process through Bayesian analysis





 Evaluate the system-level impact of cyber-attacks against digital control systems





Human Factors Engineering (HFE)

- Research on integrating human crews into complex systems
- Develop HFE guidelines for the review of human-system interface designs
- Perform safety reviews of new and modernized plant designs
- Use of technology improving crew performance and reliability





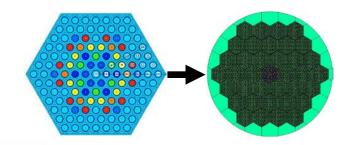


International Regulatory Assistance

- International program to improve nuclear safety in Eastern Europe → Ukraine, Armenia
- Provide nuclear regulatory support for countries establishing new nuclear programs – Poland, Vietnam, and Thailand
- Program Categories
 - Plant Safety Analysis FSARs, PRA
 - Regulatory/Legislative Activities Licensing support
- Implementation
 - Training in regulatory methods and procedures, and inspection techniques
 - Transferring analytical techniques
 - Providing and training use of equipment



VVER-440 Spent Fuel Isotopic Benchmark







Seismic/Technical Assistance to NRC

- Technical Review of Design Certification Applications;
 Evaluation of Structures, Fuel Racks, Piping, License Renewal Applications (2001 to 2008)
- Assistance in developing improved regulatory guidance for design/analysis of Seismic Cat I SSCs
- Spent Fuel Storage Cask Accidental Drop Analysis



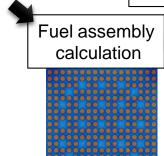


Support for DOE-NE

National Nuclear Data Center
Basic nuclear database

Thermal properties Constitutive relations

 Evaluation of advanced nuclear systems (reactors and accelerator-driven-systems (ADS) and fuels cycles



Reactor core calculation



- Advanced Fuels, Fuel Cycle Options, Material Protection Accounting & Control Technologies
- Nuclear Fuel Cycle & Supply Chain -Systems Analysis & Integration, Advanced Fuels, and LWR Modeling & Analysis
- Advanced Reactor Technologies -Proliferation Resistance & Physical Protection

