



# Radioactive Source Term for Normal Operation of Light Water Reactors

An American National Standard



## American National Standard Radioactive Source Term for Normal Operation of Light Water Reactors

Secretariat

American Nuclear Society

Prepared by the American Nuclear Society Standards Committee Working Group ANS-18.1

Published by the American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60526 USA

Approved November 1, 2016 by the American National Standards Institute, Inc.

#### American National Standard

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to ensure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) should be sent to the Scientific Publications and Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by the

American Nuclear Society 555 North Kensington Avenue La Grange Park, Illinois 60526 USA



#### This document is copyright protected.

Copyright © 2016 by American Nuclear Society. All rights reserved.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-18.1-2016 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

#### Inquiry Requests

The American Nuclear Society (ANS) Standards Committee will provide responses to inquiries about requirements, recommendations, and/or permissive statements (i.e., "shall," "should," and "may," respectively) in American National Standards that are developed and approved by ANS. Responses to inquiries will be provided according to the Policy Manual for the ANS Standards Committee. Nonrelevant inquiries or those concerning unrelated subjects will be returned with appropriate explanation. ANS does not develop case interpretations of requirements in a standard that are applicable to a specific design, operation, facility, or other unique situation only and therefore is not intended for generic application.

Responses to inquiries on standards are published in ANS's magazine, *Nuclear News*, and are available publicly on the ANS Web site or by contacting the ANS Scientific Publications and Standards Department.

### **Inquiry** Format

Inquiry requests shall include the following:

- (1) the name, company name if applicable, mailing address, and telephone number of the inquirer;
- (2) reference to the applicable standard edition, section, paragraph, figure, and/or table;
- (3) the purpose(s) of the inquiry;
- (4) the inquiry stated in a clear, concise manner;
- (5) a proposed reply, if the inquirer is in a position to offer one.

Inquiries should be addressed to:

American Nuclear Society ATTN: Scientific Publications and Standards Department 555 N. Kensington Avenue La Grange Park, IL 60526

or standards@ans.org

#### **Foreword**

(This foreword is not a part of American National Standard, "Radioactive Source Term for Normal Operation of Light Water Reactors," ANSI/ANS-18.1-2016.)

The purpose of this standard is to provide a set of typical radionuclide concentrations for estimating the radioactivity in the principal fluid streams of a light water reactor. Some systems will have different concentrations from those indicated in this standard. The values in this standard were those judged to be representative concentrations in a light water reactor over its lifetime based upon the data currently available. It is not intended that these data be used as the sole basis for design but be used in environmental reports and elsewhere where expected operating conditions over the life of the plant would be appropriate. The data and methodology provided by previous versions of this standard have been incorporated in the GALE computer codes (see Bibliography) used for the calculation of gaseous and liquid effluents from light water reactors. The changes included in this standard should be considered in future updates of these codes.

This standard is Revision 3 of American National Standard N237-1976 (ANS-18.1-1976), "Source Term Specification," and updates the default activity concentrations and adjustment factors associated with Revision 2 (ANSI/ANS-18.1-1999, "Radioactive Source Term for Normal Operation of Light Water Reactors"), based on the latest review of data from operating domestic nuclear power plants as documented in Electric Power Research Institute DRAFT Technical Document, "Technical Bases for Update of the ANSI/ANS-18.1-1999 Standard to Incorporate Contemporary Best-Estimate Radiological Source Terms in Principal Fluid Streams of Light Water Reactors" (October 2015). The values given in this standard will be revised periodically as additional plant operating data become available.

This standard does not incorporate the concepts of generating risk-informed insights, performance-based requirements, or a graded approach to quality assurance. The user is advised that one or more of these techniques could enhance the application of this standard.

This standard was prepared by the ANS-18.1 Working Group of the American Nuclear Society. The following members contributed to this standard:

- K. Geelhood (Chair), Pacific Northwest National Laboratory
- L. Benevides, U.S. Nuclear Regulatory Commission
- E. Dickson, U.S. Nuclear Regulatory Commission
- C. Fung Poon, GE Hitachi Nuclear Energy
- D. Hussey, Electric Power Research Institute
- M. O'Connor, Electric Power Research Institute
- M. Shaver, NuScale Power Inc.
- T. Lloyd, Westinghouse Electric Company

The Large Light Water Reactor & Reactor Auxiliary Systems Design Subcommittee had the following membership at the time of its approval of this standard:

- D. Kanuch (Chair), Siemplekamp Nuclear Services, Inc.
- E. Johnson-Turnipseed, *Entergy*
- L. Kreider, Engineering Planning & Management, Inc.
- M. Linn, Oak Ridge National Laboratory

The Large Light Water Reactor Consensus Committee had the following membership at the time of its approval of this standard:

- C. E. Carpenter (Chair), U.S. Nuclear Regulatory Commission
- W. Reuland (Vice Chair), Individual
- J. Bonfiglio, Florida Power & Light
- C. Brown, Southern Nuclear Operating Company
- L. Christensen, Bechtel Corporation
- J. Florence, Nebraska Public Power District
- D. Gardner, Enercon Services Inc.
- S. Gebers, Quantum Nuclear Services
- J. Glover, Graftel, Inc.
- P. Guha, U.S. Department of Energy
- E. Johnson-Turnipseed, Entergy
- L. Kreider, Engineering Planning & Management, Inc.
- M. Linn, Oak Ridge National Laboratory
- E. Lloyd, Exitech Corporation
- R. Markovich, Contingency Management Consulting
- R. McFetridge, Westinghouse Electric Company, LLC
  - (Alt. G. Corpora, Westinghouse Electric Company, LLC)
- T. Meneely, Westinghouse Electric Company, LLC
- C. Moseley, Jr., ASME NQA Liaison
- J. Riley (Liaison), Nuclear Energy Institute
- S. Routh, Bechtel Power Corporation
- R. M. Ruby (Observer), Individual
- J. Saldarini, Bechtel Power Corporation
- D. J. Spellman, *Individual*
- S. L. Stamm, Individual

#### **Contents**

Section			Page	
1		pplication, and purpose		
		<u>, , , , , , , , , , , , , , , , , , , </u>	1	
	1.2 Appli		1	
	•	se		
2		ns		
	2.1 Shall,	should, and may	1	
3	Specifica	tions	2	
		fication of radionuclide concentrations for reference plants		
	3.2 Adjus	stment of the reference plant radionuclide concentrations to a specific plant	2	
Ta	bles			
	Table 1	Parameters used to describe the reference BWR		
	Table 2	Parameters used to describe the reference PWR with U-tube steam generators	3	
	Table 3	Parameters used to describe the reference PWR with once-through steam	1	
	Table 4	Description of the element classification used in this standard		
	Table 5	Numerical values – concentrations in principal fluid streams of the reference		
	Table 3	BWR (μCi/g)	5	
	Table 6	Numerical values – concentrations in principal fluid systems of the reference		
		PWR with U-tube steam generators (μCi/g)	6	
	Table 7	Numerical values – concentrations in principal fluid streams of the reference		
		PWR with once-through steam generators (μCi/g)	7	
	Table 8	Values used in determining adjustment factors for BWRs	8	
	Table 9	Values used in determining adjustment factors for PWRs		
	Table 10	Adjustment factors for BWRs		
	Table 11	Adjustment factors for PWRs with U-tube steam generators		
	Table 12	Adjustment factors for PWRs with once-through steam generators	13	
Fi	gures			
1, 1	Figure 1	Removal paths for the reference BWR	1.4	
	Figure 2	Removal paths for the reference PWR with U-tube steam generators		
	Figure 3	Removal paths for the reference PWR with once-through steam generators		
	115010 3	The same for the reference 1 with once through steam generators	10	
Ar	pendix			
-	Bibliogra	phy	17	