American Nuclear Society

REAFFIRMED

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quality control for plate-type uranium-aluminum fuel elements

an American National Standard

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This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.



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American National Standard Quality Control for Plate-Type Uranium-Aluminum Fuel Elements

Secretariat
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American National Standard

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Foreword

(This Foreword is not part of American National Standard Quality Control for Plate-Type Uranium-Aluminum Fuel Elements, ANSI/ANS-15.2-1999.)

The American Nuclear Society Standards Committee established Subcommittee ANS-15 in the fall of 1970 with the task of preparing a standard for the operation of research reactors. In January 1972, this charter was expanded to include the task of preparing standards for all aspects of research reactor needs. To implement this enlarged responsibility, a number of working groups were established by the subcommittee to develop standards for consideration and complementary action by ANS-15. This standard addresses itself to quality control for plate-type uranium-aluminum fuel elements.

In this process of creating standards against the background of established and varied practices in many operating facilities, it is important to consider that:

- It is not intended that the standard be used as a demand model for backfitting purposes.
- b. The standard should be a vital aid for the new owner-agency.
- c. The standard should be helpful for the facility undergoing change or modification.
- Thoughtful use of the standard by industry should ease the burden of licensing and chartering agencies.

It is affirmed further that the use of any standard of performance, conduct, or excellence is volitional. The decision to use a standard is a management matter, presumably based on technical advisement.

Further guidance may be found in the following American National Standards which, like ANSI/ANS-15.2-1999, have been developed for research reactors:

ANSI/ANS-15.1-1990, Development of Technical Specifications for Research Reactors

ANSI/ANS-15.4-1988, Selection and Training of Personnel for Research Reactors

ANSI/ANS-15.7-1977 (R1986), Research Reactor Site Evaluation

ANSI/ANS-15.8-1976 (R1995), Quality Assurance Program Requirements for Research Reactors

ANSI/ANS-15.10-1994, Decommissioning of Research Reactors

ANSI/ANS-15.11-1993, Radiation Protection at Research Reactor Facilities

ANSI/ANS-15.15-1978 (R1986), Criteria for the Reactor Safety Systems of Research Reactors

ANSI/ANS-15.16-1982 (R1988), Emergency Planning for Research Reactors

ANSI/ANS-15.17-1982 (R1987), Fire Protection Program Criteria for Research Reactors

ANSI/ANS-15.19-1991, Shipment and Receipt of Special Nuclear Material by Research Reactors

At the time of the development of this revision of the Standard, Working Group ANS-15.2 had the following membership:

M. L. Gildner, Chairman, Oak Ridge National Laboratory

R. W. Knight, Oak Ridge National Laboratory

K. R. Catlett, Babcock & Wilcox

R. R. Hobbins, Idaho National Engineering Laboratory

J. C. Ottone, CERCA

Subcommittee ANS-15, Operation of Research Reactors, had the following membership at the time of this standard:

W. J. Richards, Chairman, U.S. Department of Defense A. Adams, Jr., U.S. Nuclear Regulatory Commission T. L. Bauer, University of Texas S. K. Bhatnagar, U.S. Department of Energy L. M. Bobek, Worcester Polytechnic Institute W. J. Brynda, Brookhaven National Laboratory A. F. DiMeglio, Individual P. C. Ernst, Individual J. P. Farrar, University of Virginia D. E. Feltz, Individual M. L. Gildner, Oak Ridge National Laboratory D. R. Harris, Rensselaer Polytechnic Institute D. E. Hughes, Pennsylvania State University E. Lee, Oak Ridge National Laboratory R. E. Malenfant, Los Alamos National Laboratory R. C. Nelson, Jason Associates Corporation P. B. Perez, North Carolina State University T. M. Raby, National Institute of Standards and Technology J. Razvi, GA Technologies, Inc. T. R. Schmidt, Sandia National Laboratory M. H. Voth, Pennsylvania State University R. R. Walston, U.S. Department of Energy

Consensus Committee N17, Research Reactors, Reactor Physics, and Radiation Shielding, had the following membership at the time it reviewed and approved this standard:

T. M. Raby, Chairman A. Weitzberg, Secretary

A. D. Calihan R. E. Carter D. Cokinos A. De La Paz B. Dodd D. Duffey H. Goldstein W. A. Holt W. C. Hopkins L. I. Kopp J. Miller J. E. Olhoeft T. M. Raby W. J. Richards M. M. Ter Pogossian J. F. Torrence	U. S. Department of Energy Individual Individual Individual Brookhaven National Laboratory Vista Technologies Health Physics Society American Institute of Chemical Engineers American Physical Society American Public Health Association Bechtel Corporation Individual Institute of Electrical and Electronics Engineers, Inc. Individual American Nuclear Society U. S. Department of Defense American College of Radiology National Institute of Standards and Technology
D. K. Trubey	Individual
S. H. Weiss	U. S. Nuclear Regulatory Commission
A. Adams, Jr. (Alt.)	
W. L. Whittemore	GA Technologies, Inc.

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