

ERRATUM ISSUED

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American Nuclear Society

design criteria for consolidation of LWR spent fuel

an American National Standard

REAFFIRMED

July 7, 2016

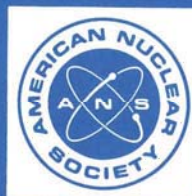
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(R2006)

This standard has been reviewed and reaffirmed with the recognition that it may reference other standards and documents that may have been superseded or withdrawn. The requirements of this document will be met by using the version of the standards and documents referenced herein. It is the responsibility of the user to review each of the references and to determine whether the use of the original references or more recent versions is appropriate for the facility. Variations from the standards and documents referenced in this standard should be evaluated and documented.

This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.



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ERRATA

ANSI/ANS-57.10-1996; R2006 Design Criteria for Consolidation of LWR Spent Fuel

The following typographical errors have been identified in this standard:

Reference [1] ANSI/ANS-57.7-1992 should be ANSI/ANS-57.7-1988; R1997

Reference [3] ANSI/ANS-57.2-1992 should be ANSI/ANS-57.2-1983

Reference [9] ANSI/ANS-8.17-1989 should be ANSI/ANS-8.17-1984; R1997

Please note that subsequent to the issuance of this standard, all three of the above standards were withdrawn. ANSI/ANS-57.7-1988; R1997 and ANSI/ANS-57.2-1983 were administratively withdrawn by the American National Standards Institute for noncompliance of maintenance within the required ten year period following approval. ANSI/ANS-8.17-1984; R1997 was superseded by ANSI/ANS-8.17-2004. The user is advised to review each reference to determine whether it, a more recent version, or a replacement document is the most pertinent for each application. When alternate documents are used, the user is advised to document this decision and its basis.

For future clarification, contact the ANS Standards Administrator at standards@ans.org.

March 2008

**American National Standard
Design Criteria for Consolidation
of LWR Spent Fuel**

Secretariat
American Nuclear Society

Prepared by the
**American Nuclear Society
Standards Committee
Working Group ANS-57.10**

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Approved May 7, 1996
by the
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American National Standard

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Comments on this standard are encouraged and should be sent to Society Headquarters.

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Foreword

(This Foreword is not a part of American National Standard Design Criteria for Consolidation of LWR Spent Fuel, ANSI/ANS-57.10-1996.)

This standard provides design criteria for the equipment and systems comprising the rod consolidation process for commercial light water reactor (LWR) spent fuel assemblies. The criteria are applicable to wet and dry, and horizontal and vertical, consolidation concepts.

The standard does not include storage of the spent nuclear fuel either prior to consolidation or upon completion of the process. There is a section in the standard which identifies interface considerations of the process with the facility or installations in which consolidation will take place.

The rod consolidation process is intended to produce canisters filled with full-length fuel rods that have been removed from spent nuclear fuel. The process removes those components that maintain rod spacing and, thereby, allows the individual fuel rods to be reconfigured into a close packed array. This is intended to result in more efficient spent fuel management.

The standard is intended to be consistent with the requirements of the regulations in Title 10, "Energy," Code of Federal Regulations, Part 50, "Domestic Licensing of Production and Utilization Facilities," and Part 72, "Licensing Requirements for the Storage of Spent Fuel in an Independent Spent Fuel Storage Installation (ISFSI)."

The revision was part of periodic, routine maintenance. It includes updated references and clarifications.

The membership of Working Group ANS-57.10 of the Standards Committee of the American Nuclear Society, during the development of this revision, was as follows:

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M. Cunningham, *Battelle Pacific Northwest Laboratories*
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F. C. Sturz, *U.S. Nuclear Regulatory Commission*
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G. A. Townes, *BE, Inc.*

Also contributing directly to the development of this revision by providing text, review, and comment was:

G. Garner, *B&W Fuel Company*

The American Nuclear Society's Nuclear Power Plant Standards Committee (NUPPSCO) had the following membership at the time of its approval of this standard:

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