

# American Nuclear Society

**calculation and measurement of direct and scattered  
gamma radiation from LWR nuclear power plants**

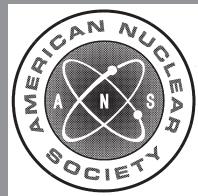
**an American National Standard**

**REAFFIRMED**

**March 5, 2007**

**ANSI/ANS-6.6.1-1987; R1998;  
R2007**

This standard has been reviewed and reaffirmed with the recognition that it may reference other standards and documents that may have been superseded or withdrawn. The requirements of this document will be met by using the version of the standards and documents referenced herein. It is the responsibility of the user to review each of the references and to determine whether the use of the original references or more recent versions is appropriate for the facility. Variations from the standards and documents referenced in this standard should be evaluated and documented. This standard does not necessarily reflect recent industry initiatives for risk informed decision-making or a graded approach to quality assurance. Users should consider the use of these industry initiatives in the application of this standard.



**published by the  
American Nuclear Society  
555 North Kensington Avenue  
La Grange Park, Illinois 60526 USA**

**ANSI/ANS-6.6.1-1987 (R1998)**  
Revision of  
**ANSI/ANS-6.6.1-1979**

**American National Standard  
for Calculation and Measurement of Direct and Scattered  
Gamma Radiation from LWR Nuclear Power Plants**

Secretariat  
**American Nuclear Society**

Prepared by the  
**American Nuclear Society**  
**Standards Committee**  
**Working Group ANS-6.6.1**

Published by the  
**American Nuclear Society**  
**555 North Kensington Avenue**  
**La Grange Park, Illinois 60526 USA**

Approved June 24, 1987; Reaffirmed January 9, 1998  
by the  
**American National Standards Institute, Inc.**

## **American National Standard**

Designation of this document as an American National Standard attests that the principles of openness and due process have been followed in the approval procedure and that a consensus of those directly and materially affected by the standard has been achieved.

This standard was developed under the procedures of the Standards Committee of the American Nuclear Society; these procedures are accredited by the American National Standards Institute, Inc., as meeting the criteria for American National Standards. The consensus committee that approved the standard was balanced to assure that competent, concerned, and varied interests have had an opportunity to participate.

An American National Standard is intended to aid industry, consumers, governmental agencies, and general interest groups. Its use is entirely voluntary. The existence of an American National Standard, in and of itself, does not preclude anyone from manufacturing, marketing, purchasing, or using products, processes, or procedures not conforming to the standard.

By publication of this standard, the American Nuclear Society does not insure anyone utilizing the standard against liability allegedly arising from or after its use. The content of this standard reflects acceptable practice at the time of its approval and publication. Changes, if any, occurring through developments in the state of the art, may be considered at the time that the standard is subjected to periodic review. It may be reaffirmed, revised, or withdrawn at any time in accordance with established procedures. Users of this standard are cautioned to determine the validity of copies in their possession and to establish that they are of the latest issue.

The American Nuclear Society accepts no responsibility for interpretations of this standard made by any individual or by any ad hoc group of individuals. Requests for interpretation should be sent to the Standards Department at Society Headquarters. Action will be taken to provide appropriate response in accordance with established procedures that ensure consensus on the interpretation.

Comments on this standard are encouraged and should be sent to Society Headquarters.

Published by

**American Nuclear Society  
555 North Kensington Avenue, La Grange Park, Illinois 60526 USA**

Copyright © 1987 by American Nuclear Society.

Any part of this standard may be quoted. Credit lines should read "Extracted from American National Standard ANSI/ANS-6.6.1-1987 with permission of the publisher, the American Nuclear Society." Reproduction prohibited under copyright convention unless written permission is granted by the American Nuclear Society.

Printed in the United States of America

## **Foreword**

(This Foreword is not a part of American National Standard for Calculation and Measurement of Direct and Gamma Radiation from LWR Nuclear Power Plants, ANSI/ANS-6.6.1-1987.)

In mid 1973, a need for a standard on this subject was identified by D. K. Trubey, chairman of ANS-6. The proposed standard had been listed among those having a high priority by the Atomic Energy Commission Directorate of Regulatory Standards. The project was assigned by the NTAB Executive Committee in September 1973. Working Group ANS-6.6 was formed and E. A. Warman was appointed chairman in September 1973. The first meeting of the working group was held in November 1973. Twelve subsequent meetings were held from February 1974 through June 1978.

The first working draft was completed in June 1975. A revised working draft was distributed for initial review by ANS-6 chairmen in June 1976. The completed Draft 1 was submitted to ANS-6 for ballot in September 1977. This draft was unanimously approved in subsequent balloting by ANS-6 subcommittee chairmen, with the sole negative ballot being changed to affirmative after resolution of comments. A reorganized working group, ANS-6.6.1, was established in 1984 to reconsider the status of the standard. In December 1985, that working group recommended reaffirmation of the standard with minor editorial changes.

In preparing this standard, the working group decided to provide a series of reference calculations with which a radiation analyst should compare results obtained by the method he elected to use in a given application. Comparison with the results of these reference calculations is intended to provide some assurance that the methods being considered by the user of this standard produce results which are in reasonable agreement with those of other methods. These reference calculations are intentionally simplistic to make this comparison effort easier to accomplish.

This standard addresses contained sources of direct and scattered radiation and specifically excludes effluent releases and accident sources. Measurements at some operating plants, which have no local shielding to reduce reactor cavity/nozzle inspection port streaming, have indicated that localized streaming can be measurable outside the containment. Such localized streaming effects are not addressed in this standard.

Particular emphasis is placed on the direct and scattered radiation from  $^{16}\text{N}$  sources in Boiling Water Reactors (BWRs). This emphasis reflects the fact that analysis and measurement of radiation associated with  $^{16}\text{N}$  sources at BWRs was identified as a major area of interest in establishing priority for development of this standard. The three appendices to the standard are included as examples of the type of measurements and analyses which have been performed in connection with the  $^{16}\text{N}$  sources at BWRs. In Appendices 1 and 2, the assumption is made that the observed dose rates are entirely due to  $^{16}\text{N}$  activity. The net effect of this assumption is to increase the amount of conservatism in the quantification of the source terms, in that other radiations are included in the measurements from which the  $^{16}\text{N}$  source terms are developed.

Working Group ANS-6.6.1 of the ANS Standards Committee had the following membership:

R. E. Faw, Chairman, *Kansas State University*  
J. A. Broadway, *U.S. Environmental Protection Agency*  
J. Celnik, *Stone & Webster Engineering Corporation*  
F. J. Congel, *U.S. Nuclear Regulatory Commission*  
K. O'Brien, *U.S. Department of Energy*

J. V. Pace III, *Martin-Marietta Energy Systems, Inc.*  
J. K. Shultis, *Kansas State University*  
Shiaw-der Su, *GA Technologies, Inc.*  
M. B. Wells, *Radiation Research Associates*  
N. B. Willoughby, *Consultant*

The membership of Subcommittee ANS-6, Radiation Protection and Shielding, at the time of its approval of this standard was:

D. K. Trubey, Chairman, <i>Oak Ridge National Laboratory</i>	D. R. Harris, <i>Rensselaer Polytechnic Institute</i>
E. T. Boulette, <i>Maine Yankee Atomic Power Company</i>	W. C. Hopkins, <i>Bechtel Corporation</i>
J. C. Celnik, <i>Stone &amp; Webster Engineering Company</i>	E. Normand, <i>Boeing Aerospace Company</i>
	P. J. Persiani, <i>Argonne National Laboratory</i>
	D. J. Schuh II, <i>GEB Controls Group, Inc.</i>

Consensus Committee N17, Research Reactors, Reactor Physics, and Radiation Shielding, had the following membership at the time it reviewed and approved this standard:

R. S. Carter, Chairman  
T. M. Raby, Secretary

J. D. Buchanan	Health Physics Society
A. D. Callihan (Subcommittee ANS-1)	Individual
R. E. Carter	U.S. Nuclear Regulatory Commission
C. Thomas (Alt.)	
R. S. Carter	American Nuclear Society
A. De La Paz (Subcommittee ANS-14)	Department of U.S. Army
D. Duffey	American Institute of Chemical Engineers
H. Goldstein	American Physical Society
P. B. Hemming	U.S. Department of Energy
J. W. Lewellen (Alt.)	
W. A. Holt	American Public Health Association
L. I. Kopp (Subcommittee ANS-10)	U.S. Nuclear Regulatory Commission
J. E. Olhoeft	Individual
T. M. Raby	National Bureau of Standards
W. J. Richards (Subcommittee ANS-15)	Argonne National Laboratory
M. M. Ter Pogossian	American College of Radiology
D. K. Trubey (Subcommittee ANS-6)	Oak Ridge National Laboratory
A. Weitzberg (Subcommittee ANS-19)	NUS Corporation
W. L. Whittemore	Individual

<b>Contents</b>	<b>Section</b>	<b>Page</b>
1.	Scope .....	1
2.	Applicable Documents .....	1
3.	Terms and Definitions .....	1
4.	Requirements .....	2
5.	Dose Assessment Guidelines .....	3
6.	Calculations .....	4
6.1	General Introduction .....	4
6.2	Sources .....	4
6.3	Source Modeling (Mathematical Representations) .....	4
6.4	Geometric Modeling (Shielding and Plant Layout) .....	6
6.5	Site Topography .....	7
6.6	Radiation Transport Methods .....	7
7.	Measurement Constraints and Interpretation of Data .....	11
7.1	General Considerations .....	11
7.2	Measurement Techniques .....	19
7.3	Measurement Techniques Summary .....	21
8.	References .....	21
<b>Appendices</b>		
1.	Semi-Empirical Determination of $^{16}\text{N}$ Specific Activity at an Operating BWR .....	23
2.	Estimated $^{16}\text{N}$ Specific Activity From Main Steam Line Measurements at a 1,600 MWt BWR and at a 2,400 MWt BWR .....	27
3.	Measurements of $^{16}\text{N}$ Gamma Radiation Fields Near Operating BWRs .....	36
<b>Figures</b>		
Fig. 6.1	Geometrical Configurations for Reference Calculations I.1 and I.2 .....	9
Fig. 6.2	Geometrical Configurations for Reference Calculations II.1 and II.2 .....	10
Fig. 6.3	Scattered Dose Rate as a function of Horizontal Distance for Reference Problem No. I.1 .....	12
Fig. 6.4	Total Dose Rate as a function of Horizontal Distance for Reference Problem No. I.1 .....	13
Fig. 6.5	Scattered Dose Rate as a function of Horizontal Distance for Reference Problem No. I.2 .....	14
Fig. 6.6	Total Dose Rate as a function of Horizontal Distance for Reference Problem, No. I.2 .....	15
Fig. 6.7	Total Dose Rate as a function of Horizontal Distance for Reference Problem No. II.1 .....	16
Fig. 6.8	Total Dose Rate as a function of Horizontal Distance for Reference Problem No. II.2 .....	17

Fig. A-1.1	Dose Point Designations and Locations .....	25
Fig. A-2.1	Cooper Nuclear Power Station .....	30
Fig. A-2.2	Duane Arnold Energy Center .....	31
Fig. A-2.3	Steamline Radiation Monitor Readings .....	33
Fig. A-3.1	Distance From Source (meters) .....	37

Tables

Table 6.1	.....	5
Table 6.2	.....	5
Table A-1.1	.....	26
Table A-1.2	.....	26
Table A-2.1	.....	32
Table A-2.2	.....	32
Table A-2.3	.....	34
Table A-2.4	.....	34
Table A-2.5	.....	35
Table A-2.6	.....	37