

NUCLEAR REACTOR THERMAL HYDRAULICS (NURETH-10) SPECIAL ISSUE—PART 2

- 143** Foreword / *Soon Heung Chang, Won-Pil Baek, Joy Rempe*
- 145** Limiting Factors for External Reactor Vessel Cooling / *F. B. Cheung*
- 162** Critical Heat Flux Experiments on the Reactor Vessel Wall Using 2-D Slice Test Section / *Yong Hoon Jeong, Soon Heung Chang, Won-Pil Baek*
- 170** Development of an Enhanced Core Catcher for Improving In-Vessel Retention Margins / *J. L. Rempe, K. G. Condie, D. L. Knudson, K. Y. Suh, F. B. Cheung, S. B. Kim*
- 183** KAERI Integral Effect Test Program and the ATLAS Design / *Won-Pil Baek, Chul-Hwa Song, Byong-Jo Yun, Tae-Soon Kwon, Sang-Ki Moon, Sung-Jae Lee*
- 196** Evolution of Design Methodologies for Next Generation of Reactor Pressure Vessels and Extensive Role of Thermal-Hydraulic Numerical Tools / *Serge Bellet, Nicolas Goreaud, Norbert Nicaise*
- 210** Experimental Study on Flow Optimization in Upper Plenum of Reactor Vessel for a Compact Sodium-Cooled Fast Reactor / *Nobuyuki Kimura, Kenji Hayashi, Hideki Kamide, Masami Itoh, Tadashi Sekine*
- 223** Computational Study of Thermal Striping in an Upper Plenum of Kalimer / *Seok-Ki Choi, Myung-Hwan Wi, Won-Dae Jeon, Seong-O Kim*
- 239** New Challenges in Computational Thermal Hydraulics / *George Yadigaroglu, Djamel Lakehal*