SUBJECT INDEX

A
Annular fuel rods, PWR, 109-2

B
Brayton cycle, 281-3
Burnup credit, 39-1, 57-1
BWR, UNF, 322-3

C
Cask, transport UNF, 208-2
CeBr3(Ce) detector, 309-3
Combined cycle, 281-3
Computer codes
AMBER, 322-3
CASMO/SIMULATE, 39-1
CITATION, 270-3
DRAGON, 174-2
KENO, 57-1
MCNP, 57-1, 85-1
ORIGEN, 208-2, 322-3
PERKY, 270-3
RELAP5, 1-1, 227-3
SAS4A, 21-1
SCALE, 57-1, 208-2
SERPENT, 174-2
SNPRT, 270-3
TOUGH2, 322-3
TSUNAMI, 39-1
TWOTRAN, 270-3
Control decoupling SCWR, 239-3
Core
accident, 21-1
simulator, 71-1

Citations are by page number and issue

D
Detector size, gamma, 309-3

E
Enriched uranium, fuels, 1-1

F
Fast Na-cooled reactor, 21-1
Fluoride-salt-cooled reactor, 281-3
Fuel cycle, dose, 192-2
deployment, 39-1
TRISO particles, 227-3

G
Gamma detector, size effect, 309-3
Geologic repository release, 322-3

H
Heat transfer, UNF, 208-2
HEU core conversion, 1-1
HTGR, 227-3

I
IFR, 100-1

K
Kinetics, 109-2

L
LEU core conversion, 1-1
Loss of circulation, HTGR, 227-3

M
Microfluids, solution sampling, 216-2
Moderator temperature coefficient, 127-2, 147-2
Molten salt reactor, 100-1
MSBR, 100-1

N
Na-cooled reactor, 21-1
Na void coefficient, 270-3
Neutron beam facility, 85-1
Nuclear fuel, see Fuel
Nyquist array, 239-3

O
Occupational dose, 192-2

P
Pebble bed reactor, 259-3
Pebble position, 259-3
PWR
fuel depletion, 39-1
reactivity, 109-2
Th fuel, 127-2, 147-2
used fuel pool criticality, 57-1

R
Radiation dose, workers, 192-2
Radioactive solution sampling, 216-2
Radioactive waste management, 100-1
RBMK-1500, UNF, 322-3
Reactivity insertion, 1-1
Repository release, 322-3

S
Sampling, radioactive solution, 216-2
SCWR, 239-3
Simulator, core, 71-1
Sodium, see Na
Spent nuclear fuel, see UNF
Supercritical water reactor, 239-3

T
Thermal-hydraulics PWR, 109-2
Th fuel, PWR, 127-2, 147-2
TRIGA, 85-1
TRISO particles, 227-3
TRU
burning, 127-2, 270-3

U
UNF
BWR, RBMK-1500, 322-3
decay heat, 208-2

V
Viscosity, artificial, 296-3

W
WWER (VVER)-440 UNF, 208-2